



Stresses Induced by LOCA in Steam Generator Collector of the VVER1000 Reactor

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During Double End Break Loss of Coolant Accident (DEBLOCA) in the VVER-1000 reactor, in the first second, pressure waves had been noticed in all points hitting the main components of the primary cooling circuit. Studying the stresses due to DEBLOCA is important to avoid equipment failure. The main attention is focused on the pressure drops on the equipment, because this parameter determines the dynamic loads on the equipment, which can lead to its failure. ATHLET code is used to obtain pressure and temperature variation with time during the accident for the VVER1000 reactor Kalinin-3. The results are used in ANSYS finite element package to calculate stresses in the steam generator collector. The stress, just before the emergency (at time 0.014 sec), was $4.3E+7$ pa and became $4.38E+6$ pa just after the emergency (at time 0.022 second). Fluctuations appear with two peaks of highest amplitude of $1.87E+7$ and $2.38E+7$ pa at time of 0.06 and 0.094 sec respectively, which is well below the allowable stress. The results indicate the safe operation of the equipment.

Keywords: VVER1000 / LOCA / Steam Generator Collector / Stresses

Introduction

The occurrence of pressure waves and their propagation in the equipment of the primary circuit of the VVER-1000 reactor plant in emergency situations, associated with an instantaneous two-way flow with an instantaneous two-way break in the cold thread of the main circulation pipeline, has been studied [1-2].

The influence of the time of initialization of the accident on the intensity of the process-amplitude and frequency of pressure changes was investigated. Pressure drops during an emergency on the main elements of the circuit was presented.

It was shown that the maximum changes in the amplitude and frequency of both the pressure and the pressure drop on the circuit elements belong to the initial stage of the accident. The main attention is focused on the pressure drop on the equipment, because this parameter determines the dynamic loads on the equipment, which can lead to its failure.

Germany and certified in Russia for use in calculations to justify the safety of reactors with water coolant [5].

In the present paper, a study for the consequence stress on the steam generator in the first loop of the

primary circuit is considered. Finite Element package (ANSYS) is used to obtain stresses on the upper part of the cold collector. A three-dimension model is used to simulate this part.

Accident scenario:

At the first stage of modeling, the specified parameters are used by specially designed regulators. At the second stage, when specially designed regulators are disabled and real nuclear power plant regulators are used, and already use them to maintain the necessary stationary conditions with the parameters set before the transition process in the installation objects: pressures, levels, temperatures, power, etc.

Only after maintaining a certain period of the necessary stationary state, the transition process begins, caused by modeling double end break LOCA.

First, a normal operation for 1000 seconds was considered and then within $10E-4$ second a complete break "double end 80% of the cross-section area" is considered in location L4.

Explanation of the task

The work presented in this paper is a part of multi-step research which is done as follows:

1. Studying the situation of instant stop of the MCP (Main Circulation Pump) in the first loop of the primary coolant circuit. we assume an instant stop of the pump, its jamming, that is, the pump blades do not turn. In this case, the flywheel of the motor is disconnected from the pump shaft. The program has the ability to simulate a similar pump stop[2]
2. Considering LOCA (Loss of Coolant Accident) (DEB "Double End Break") in the first loop of the primary coolant circuit near to the core entrance (Location. "L1"), shown in Figure (1) [6]
3. Comparing the two emergency situations from the point of view of the consequence pressure waves in the main components of the reactor. [7,8]
4. Analysis of uncertainty in the concept of "instantaneous" in the calculation of emergency modes. Five different gaps of time were considered for both emergency cases. [9]
5. LOCA in different locations in the first loop of the primary circuit: 6 locations

(shown in Figure 1) for LOCA were considered in this step.[10]

6. The most dangerous places of LBLOCA (Large Break Loss of Coolant Accident) in the four loops of primary coolant circuit as a source of dynamic load on the equipment: The six locations in step 5 were considered in each loop. [11]
7. The effect of reactor power behavior on LBLOCA hydrodynamics. [12]

The most important common conclusion for this study was: The initial period of the accident is considered in detail because the strongest amplitude and frequency of pressure fluctuations on NPP elements is occurred, which can lead to significant dynamic loads on the structural elements of these objects. This can be estimated either by the joint strength and hydrodynamic calculations, or by the possible use of the results obtained in this study as boundary conditions for the calculation of dynamic loads. This situation was the same considering 6 different locations for the DEBLOCA, as shown in Figure (1).

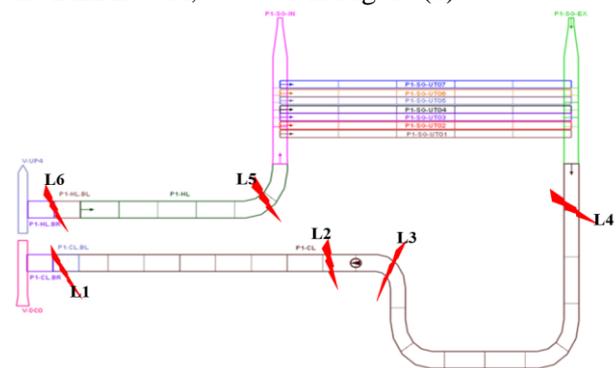


Fig.(1): The six studied locations for DEBLOCA

A Brief Description of the calculation code ATHLET

The thermo hydraulic system code ATHLET (Analysis of Thermal-hydraulics of LEaks and Transitions) was developed in plant and reactor safety society(GRS mbH) and was originally intended for analysis of the entire spectrum of leak and transient analyses in PWR and BWR reactors. However, experience with this approach has shown that it can be successfully used to the full extent for Russian reactors such as VVER and RBMK. ATHLET consists of several basic modules that allow describing different phenomena in the behaviour of thermal hydraulic systems. These are thermal hydraulic module (TFD), heat exchange

and thermal conductivity module (HECU), neutron-kinetic module (NEUKIN) to describe point and one-dimensional kinetics, module to describe the operation of the equipment (GCSM) and fully implicit module for numerical integration (FEBE). Other independent modules can be connected via the main interface.

The TFD module is based on the use of five or six equations (the General equation for the moment of motion of the mixture with the drift flow) to describe the two-fluid model. In addition to light water, the module allows calculations with other types of reactor coolants, such as: heavy water, liquid metals (lead, lead-bismuth, sodium), gases (helium). The module also simulates the behavior of non – condensable gases (air, nitrogen, argon, hydrogen, helium, oxygen, user gas-when the user sets its properties), nitrogen dissolution and describe the transport of boron in systems with light and heavy water.

The plant cooling system is modeled by connection of the basic thermodynamic objects (TFO). Besides, there is a special possibility for calculation of cross flows in system of the parallel connected channels. Quite widely used capabilities

of the ATHLET code, for linking with various three-dimensional neutron-physical programs in the calculation of the spatial distribution of energy release fields and the spatial distribution of the coolant parameters in the reactor core (up to the sub-cassette), are presented in several previous publications[13-18].

VVER1000 steam generator

The steam generator is the point where the water (coolant) from the primary (irradiated) circuit and the water from the secondary cooling circuit (clean) are contracted, but not mixed (just for a heat exchange). The structure and dimensions of the VVER-1000 steam generator is shown in Figure (2).

The point where the two cooling circuits are connected is where the steam generator linked the primary and secondary cooling water is shown in Figure (2).The upper part (cone-shaped) which is used in the finite element model to calculate the stresses included in the steam generator collector, is shown in Figure (2b).

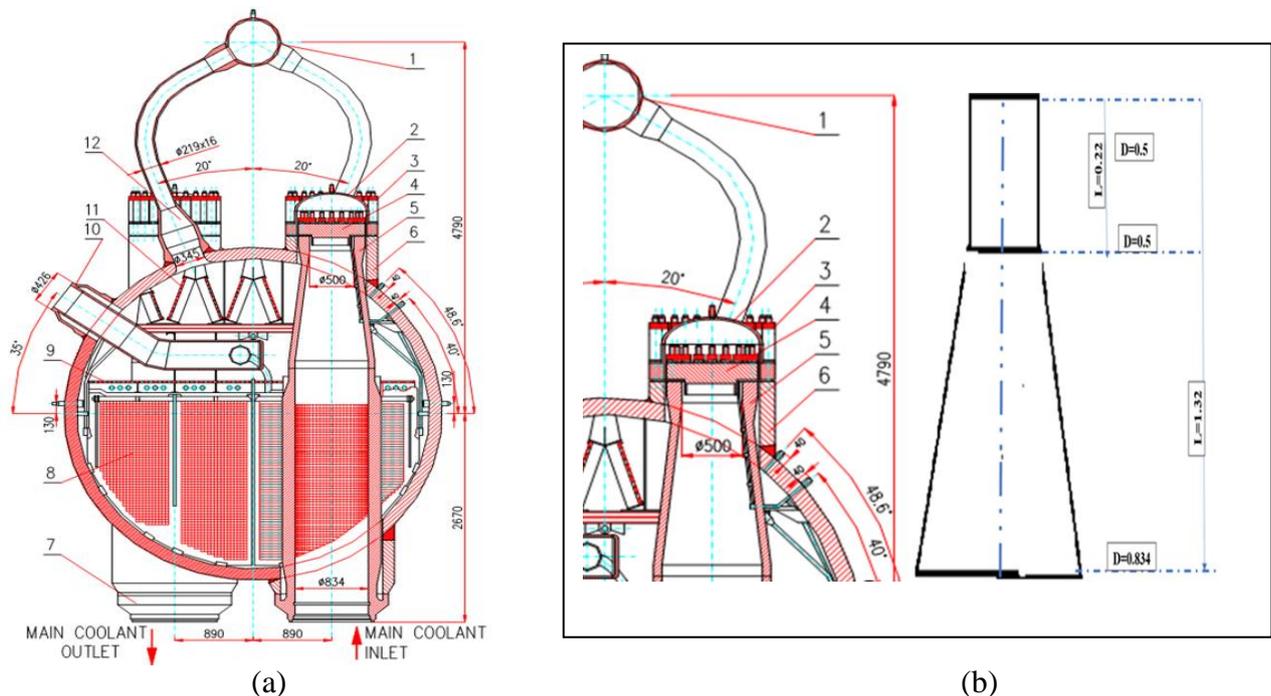


Fig. 2: (a):VVER1000 steam generator (1- Steam Header 2- Secondary Circuit (SC) Header Cover 3- Nuts 4- Primary Circuit (PC) Header Cover 5- SC Header 6- PC Header 7- MCC Nozzle 8- Heat Exchange Tubes 9- Steam Pressure Equalization Unit 10-Feedwater Nozzle 11-Separation Units 12-Steam Nozzle)
(b): Enlargement of the Upper (cone-shaped) Part of the Collector and its Cover

collector was $15.3\text{E}+6$ Pa just before the emergency (at time = 1000.004 second) and drops to $5.5\text{E}+6$ Pa just after the emergence (at time = 1000.022 second), where the pressure in the inlet point of the steam generator was $15.4\text{E}+6$ Pa just before the emergency (at time = 1000.004 second) and drops to $8.7\text{E}+6$ Pa after the emergency (at time = 1000.012 second).

The temperature change with time in °C in the exit point (P1-SG-EX) of the steam generator collector, in the Inlet point, (P1-SG-IN) of steam generator collector, and steam generator secondary circuit side (S1-UP) are shown in Figure (6). It is noticed that the temperature at the inlet point of the steam generator was 279°C (at time = 1000.002 second) and become 299°C (at time = 1000.013 second), whereas the temperature at the exit point was 279°C before the emergency (at time = 1000.01 second) and drops to $\sim 275^\circ\text{C}$ (at time = 1000.022 second) after the emergency.

The instantaneous change in the pressure and temperature with the strongest amplitude and frequency are considered a source of dynamic loads on the structural elements of these objects.

Finite Element Results

The equivalent stress [20] (σ_e) is related to the principal stresses by the following equation:

$$\sigma_e = \sqrt{\frac{(\sigma_1 - \sigma_2)^2 + (\sigma_2 - \sigma_3)^2 + (\sigma_3 - \sigma_1)^2}{2}} \quad (1)$$

The principal (normal) stresses (σ_1 , σ_2 , σ_3) are always ordered such that $\sigma_1 > \sigma_2 > \sigma_3$.

Von Mises equivalent stress is obtained through the first second during DEBLOCA. Stress fluctuations due to accidents indicate how the safety factor should be chosen to compensate for abnormal sudden change conditions.

The sudden change in the pressure and temperature with the high amplitude and frequency are considered a source of severe dynamic load. Dimensions, materials, and design of the equipment are highly sensitive to generated stress level due to accident conditions.

The dominant Von Mises stress during the analysis time, shown in Figure (7), is well below the allowable stress (295 MPa), which means that the case is in safe operation of the equipment. The upper and lower lines in Figure (7) present maximum and minimum stress variation with time respectively.

The stress just before the emergency (at time 0.014 sec) was $4.3\text{E}+7$ Pa and becomes $4.38\text{E}+6$ Pa just after the emergency (at time = 0.022 second). Fluctuations appear with two peaks of highest amplitude of $1.87\text{E}+7$ and $2.38\text{E}+7$ Pa at time of 0.06 and 0.094 sec respectively.

Even though the pressure fluctuations are not accompanied by stresses exceed the allowable stress, the pressure fluctuations can cause fatigue seed which may cause future failure. It is highly recommended to study fatigue due to such accidents.

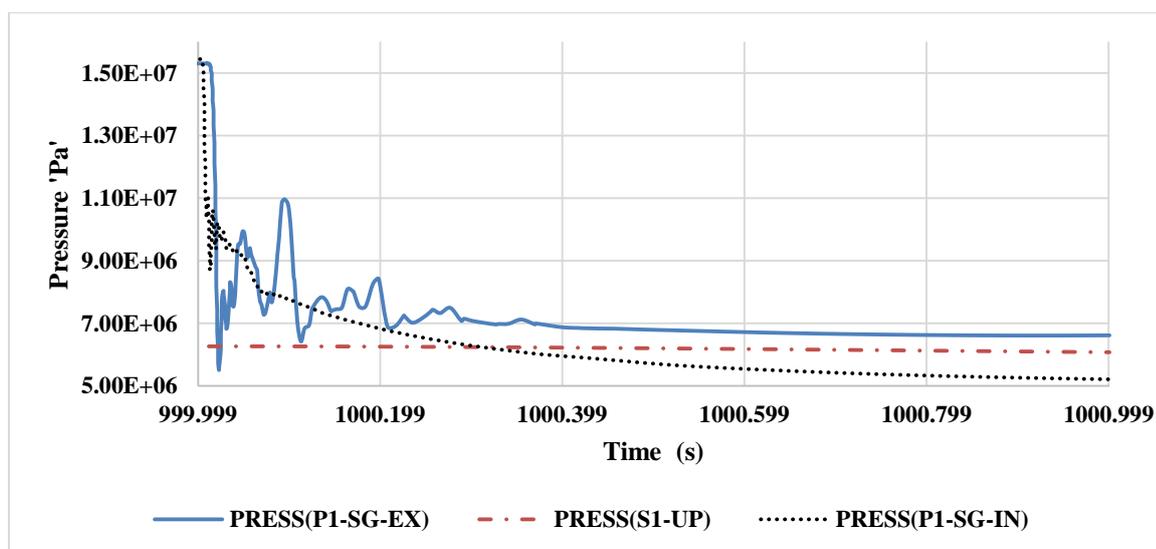


Fig. (5): Pressure change with time in: the exit point (P1-SG-EX) of steam generator collector, the Inlet point (P1-SG-IN) of steam generator collector, and steam generator secondary circuit side (S1-UP)

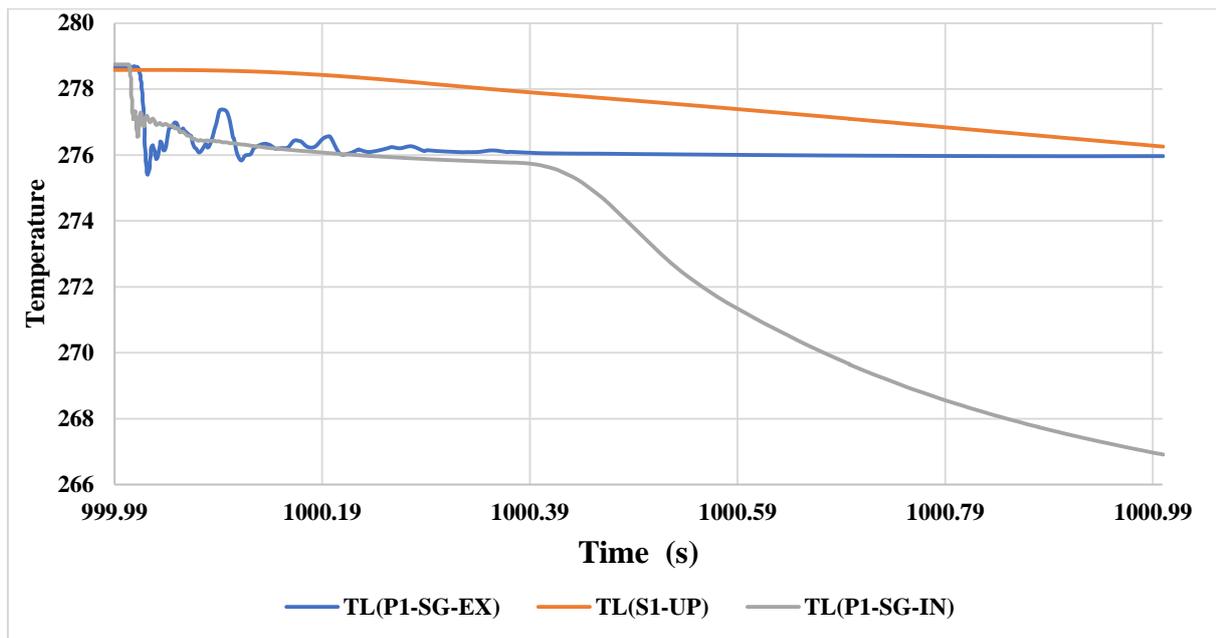


Fig.(6): Temperature change with time in: the exit point (P1-SG-EX) of steam generator collector, the Inlet point (P1-SG-IN) of steam generator collector, and steam generator secondary circuit side (S1-UP)

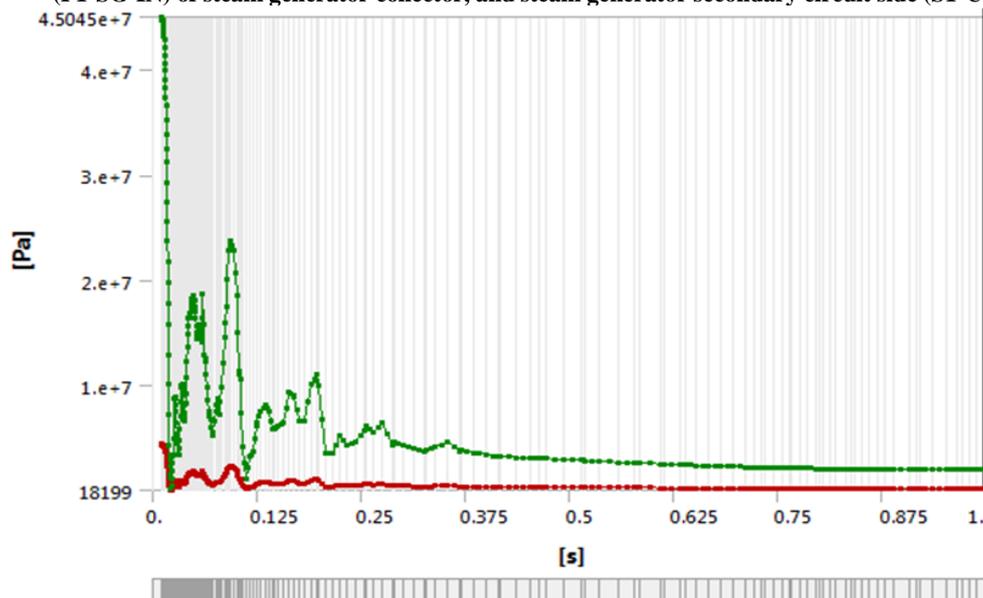


Fig.(7): Von Mises Stress variation with time

Conclusion

The initial period of the accident is analysed in detail because the high amplitude and frequency of pressure fluctuations on NPP elements have occurred, which can lead to significant dynamic loads on the structural elements of collector. The results obtained in the first part of the present work are considered as boundary conditions for the calculation of dynamic loads. Through the present work, a three-dimension model is used in ANSYS

finite element package to simulate the upper part of the cold collector where the maximum pressure waves occurred during the accident. Stresses on the simulated part are evaluated and explained. The results reflect the high safety factor of the design area in such severe accident. The maximum generated Von Mises stress is in the order of 23.8 MPa which is far away from the yield point of the operating steel which is 295 MPa.

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